# EXPLORING THE DEPLOYMENT OF ADVANCED REACTOR SYSTEMS FOR DECARBONIZATION OF FUTURE ENERGY GENERATION

Geert-Jan de Haas, Ferry Roelofs SNETP Forum, Lyon, 2. June 2022

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EU DuC = N

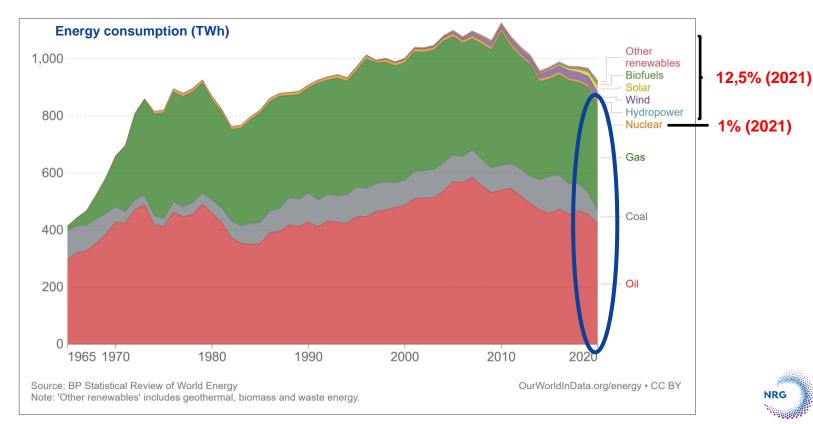


Nuclear. For life.





## The Dutch energy challenge





## **Energy functions**



**Electricity** 



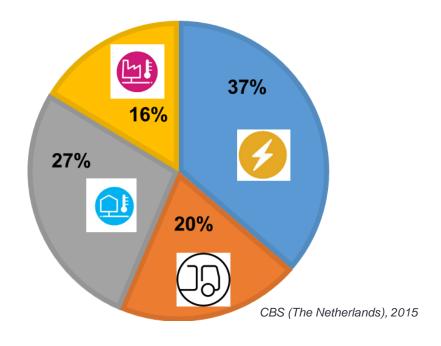
Low T heat



High T heat



Transport





## **Future energy landscape**

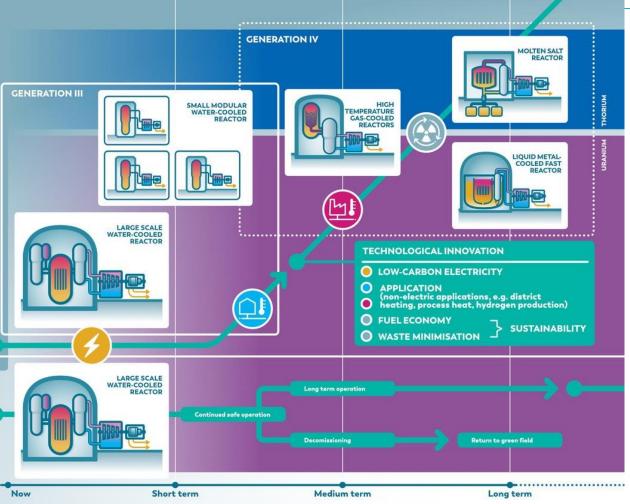
- Electrification
- Clean / carbon free
- Reliable
- Safe
- Affordable

Small footprint



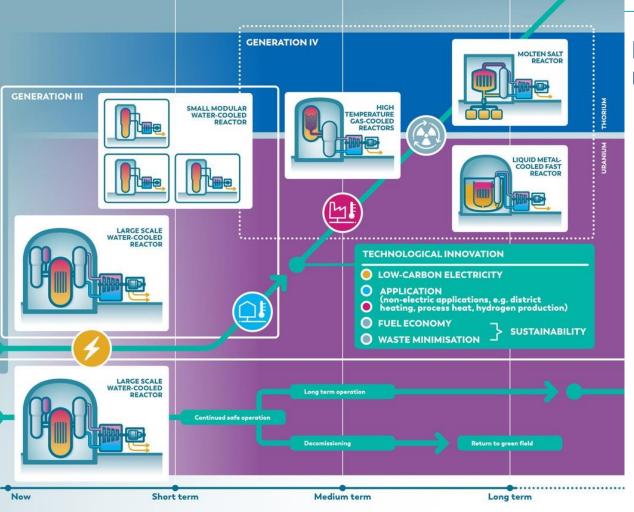
energienext.nl







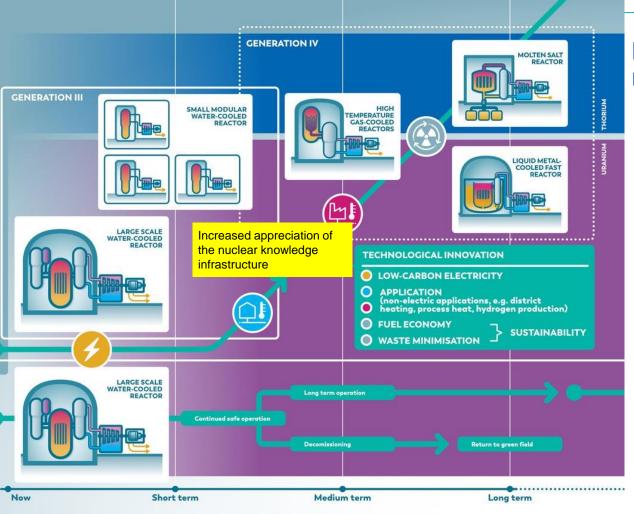






Multiannual research programma 2021-2024

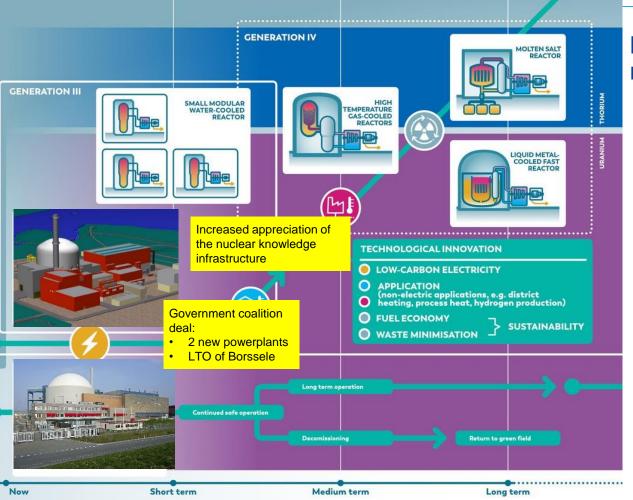






Multiannual research programma 2021-2024







Multiannual research programma 2021-2024



## Bridging the gap from R&D to deployment

- Various options to contribute to further decarbonization of the present energymix.
- Parts of the knowledge base have previously been developed.
- Fulfillment of the prospects has been pending for many decades.
- Some technologies may now be on their way to deployment.

#### MRSE ORNL (USA)



LWR-SMR, CAREM (Argentina)



BN800 (Russia)



HTR-PM (China)



#### **BUT**

- if nuclear energy is to contribute to further decarbonization .....
- if the promising prospects are really to be fulfilled....

#### THEN

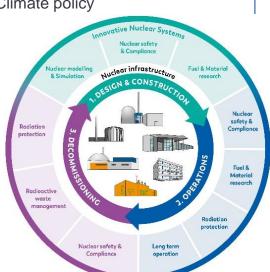
a concerted effort from the research community, policy makers, and industry is required to expand the knowledge base enabling future evaluation and assessment of the safety features by regulatory bodies.



#### **PIONEER** programme

Program for Innovation and cOmpetence development for NuclEar infrastructurE and Research

- NRG Multiannual research program 2021-2024
- Funded by Ministry of Economic Affairs and Climate policy
- 7 themes
  - 1. Long Term Operation
  - 2. Nuclear Safety and Compliance
  - B. Nuclear Modelling and Simulation
  - 4. Fuels and Materials
  - 5. Radioactive Waste Management
  - Radiation Protection
  - 7. Innovative Nuclear Systems





#### PIONEER NRG research programme 2021-2024

Commissioned by the Ministry of Economic Affairs and Climate (EZK)



NRG-24408/20.175271; 29 june 2020



## **PIONEER** contributions to Generation IV deployment

Fuels	Materials	System Simulation	Component Simulation
?	?	<b>⊘</b>	?
<b>⊘</b>	<b>⊘</b>		<b>⊘</b>
<b>⊘</b>	<b>⊘</b>		
			<b>⊘</b>



Possible future activities



Past and current activities

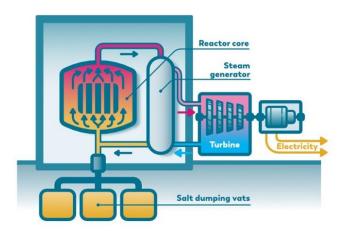


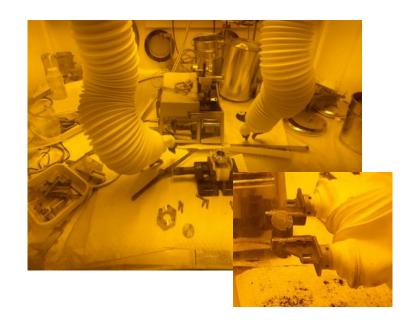
Focal point activities



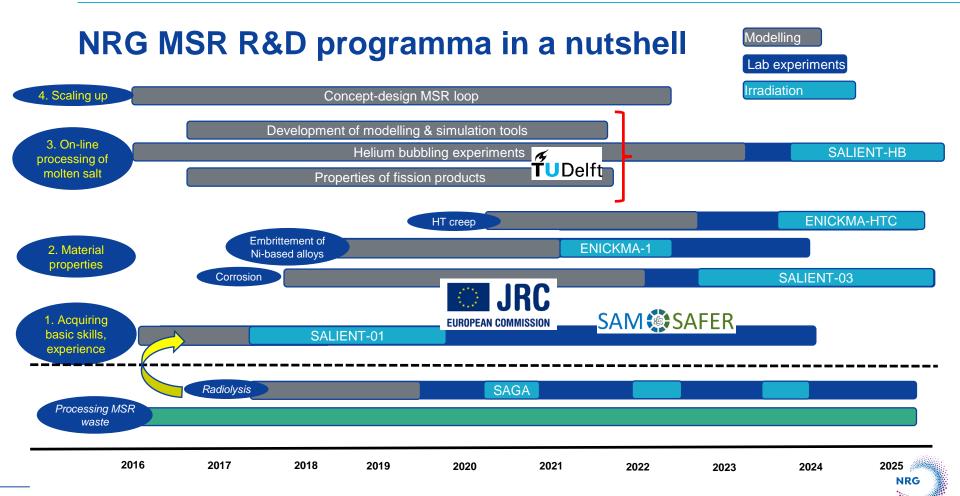
## **Highlight 1. Molten Salt Reactors**

- Continuous recycling of thorium
- Almost no high level radioactive waste
- Electricity and heat for coming centuries





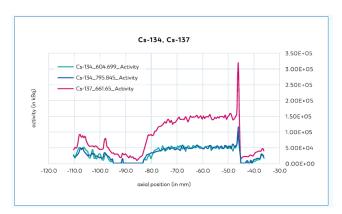




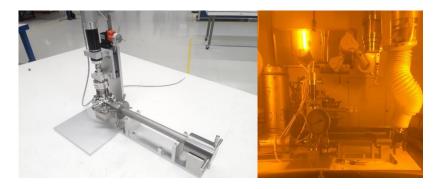
#### **MSR:** fuelled salt irradiation

Issue: hardly any experience with molten salt fuel irradiations and handling of fuelled salts

- Graphite
- Salt composition based on manufacturing possibilities (78LiF-22ThF<sub>4</sub>)
- Irradiation at two temperatures during ~500 full power days
- Determination fission gas release
- Gamma spectrometry
- Electron microscopy
- Fission gas release









## **MSR:** radiolysis

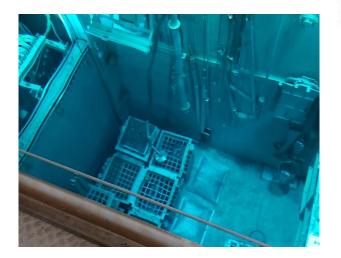
**Issue**: F<sub>2</sub> formation at lower temperatures (in-between irradiation cycles)

→ pressure build-up

SAGA: gamma irradiation fuel salt at low temperature

- HFR spent fuel as gamma source
- First irradiation completed
- Recombination tests ongoing
- Preparation second irradiation













## **MSR:** processing of waste

Any experimental programme generates new waste streams.

Lessons from the past: experimental fuel research has to be attended with research into the stabilization of waste.

#### Issues:

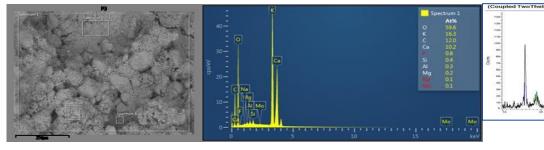
- 1) MSR waste is corrosive.
- 2) Ambient conditions will give rise to radiolysis (room temperature).

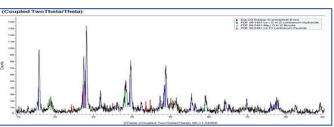
Required: onversion to stable waste forms acceptable for COVRA, Dutch facility for radioactive waste storage

Mixed fluoride salts → oxide waste and cementable liquid waste

Phase 1: simulated waste (non-active components)

Phase 2: radioactive samples from SALIENT-01 experiment in hot-cell







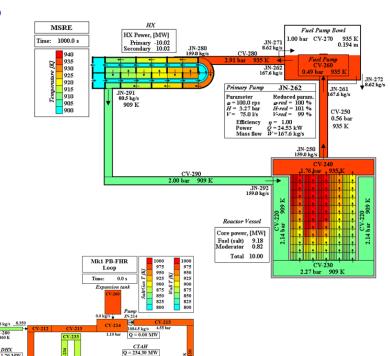
#### MSR: system thermal hydraulics

#### Molten salt model development:

- delayed neutron precursor drift
- heat removal by natural circulation
- fission product transport in (fueled) molten salt reactors
- noble gas and noble metal behavior
- noble metal extraction
- chromium leaching and deposition

Comparison and validation (whenever possible)

- MSRE
- Mk1-PB-FHR



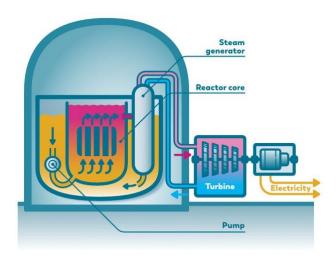
CV-290

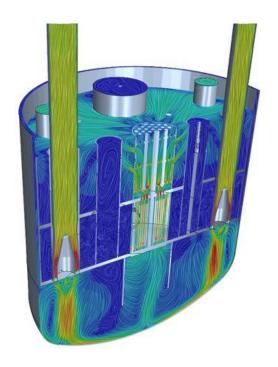
Q<sub>finid</sub>= 236.0 MW T<sub>finid</sub>= 1058.5 K



## **Highlight 2. Liquid Metal-Cooled Reactors**

- Multi-recycling of uranium
- 20x more energy from uranium
- Less high level radioactive waste
- Electricity and heat for coming centuries

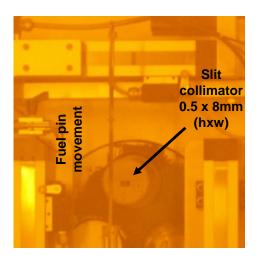


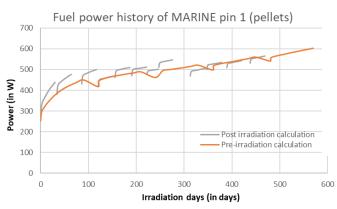


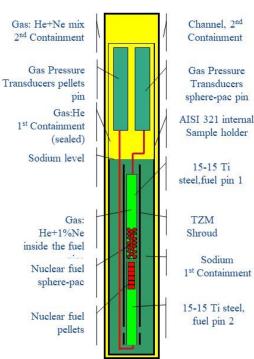


#### LMFR: transmutation of actinides

- Comparison of pellet and sphere-pac blanket fuel (U<sub>0.85</sub>Am<sub>0.15</sub>O<sub>2-x</sub>)
- Focus on He and fission gas production / release
- Measuring gas release into the pin plenum online
- Nuclear analysis and non-destructive examination
- Future activity: destructive examination

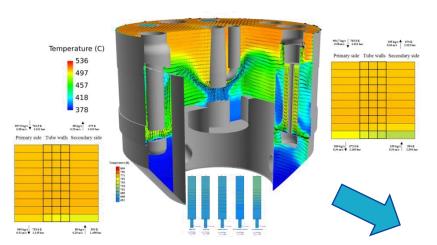








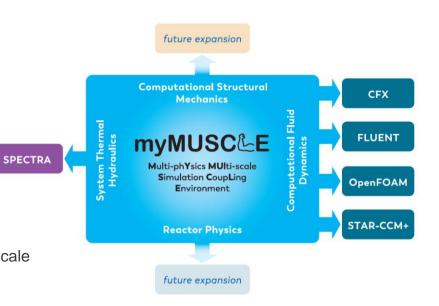
#### LMFR: multi-scale thermal hydraulics



Phénix: asymmetric transient

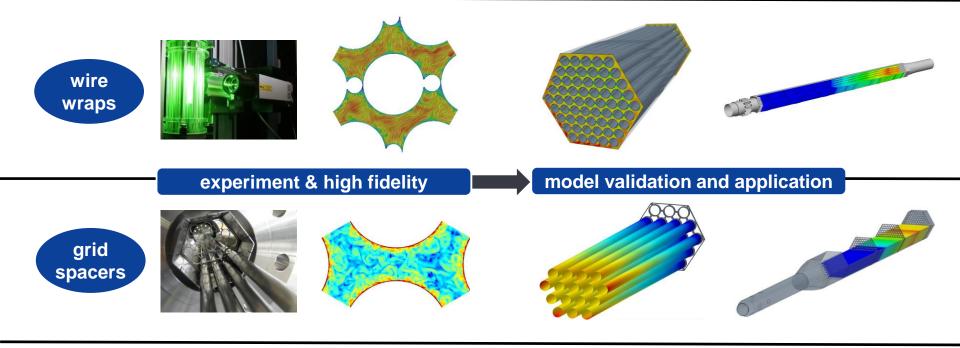
Development of myMUSCLE generic multi-scale multi-physics coupling tool:

- Coupling methods
- Coupling algorithms
- User-friendly interface





#### LMFR: core thermal hydraulics





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