

Support to MSR technology

SNETP Forum, 2-4 February 2021



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Contents

Molten Salt Reactor Developments

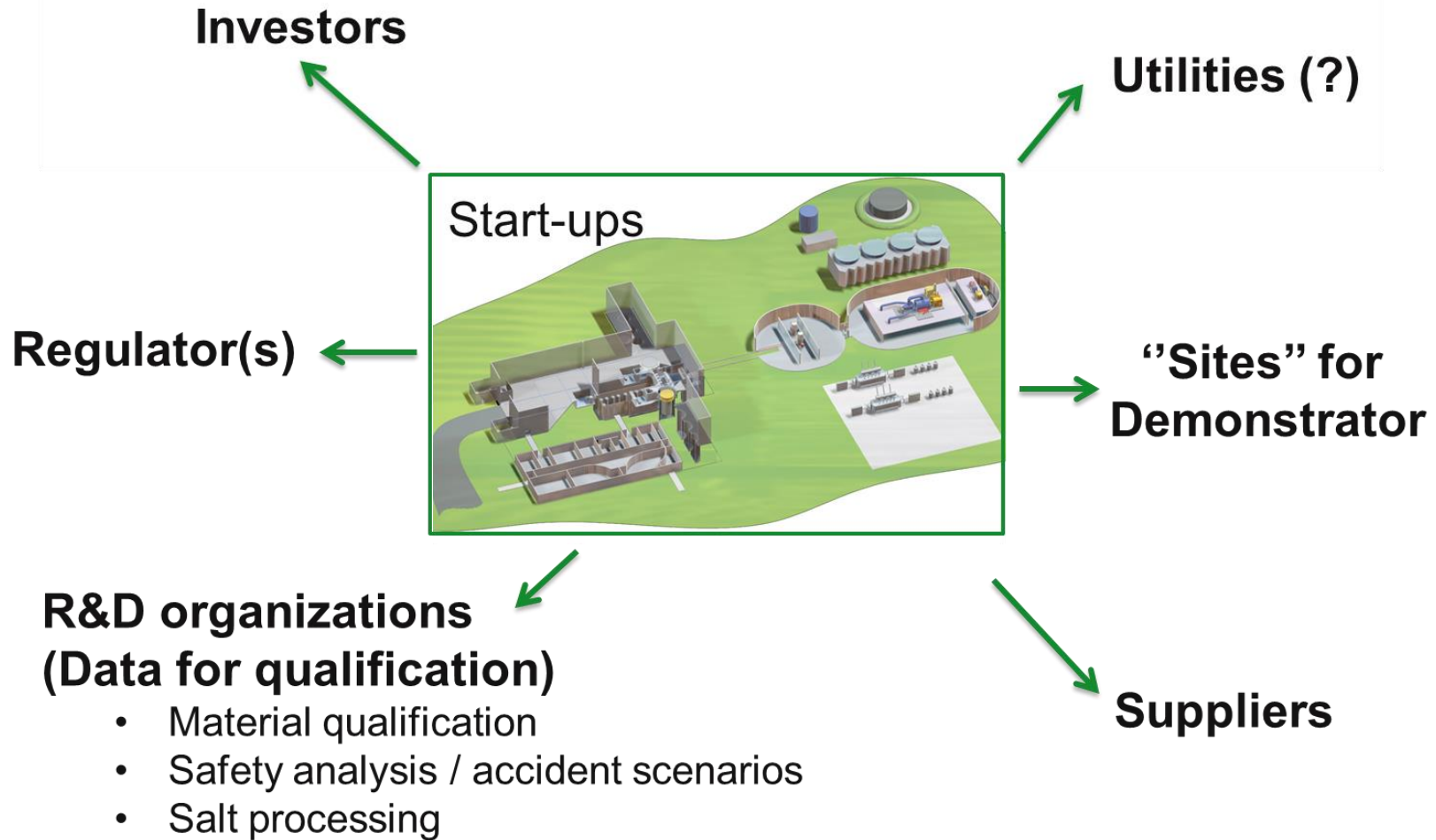
The Dutch Molten Salt Program

Irradiation program on Reactor Safety

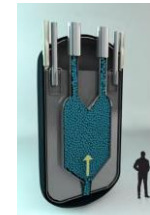
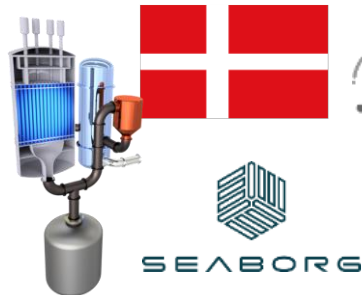
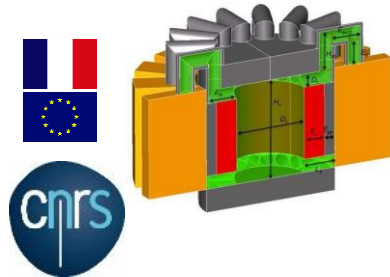
Support in modelling MSR



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Many International MSR developers



MANY MSR INITIATIVES, BUT...

1. Some seem to promise all the benefits of using thorium...

But this is complicated due to online fission product management

2. Fast track reactors have a modular approach based on MSRE design

But performance is not necessarily much better than existing solid fuel reactors
MSRE was a test reactor, not a first of a kind power system,
and new waste issues are created (what do to with the salt?)

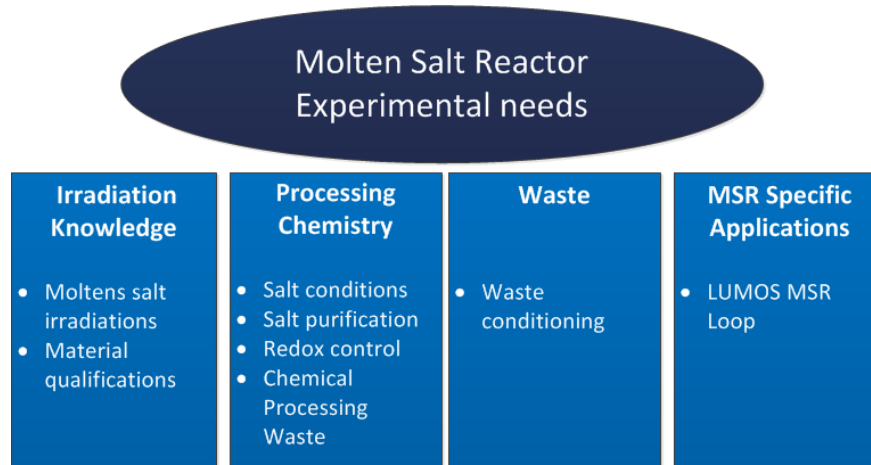
3. Fast spectrum molten salt reactors can close the U-Pu cycle

But require proof and perhaps more development than thermal MSRs
and MSR promised safety is not guaranteed

Few of them currently invest significantly in the experimental effort needed for design, safety evaluation, code and system validation, and licensing

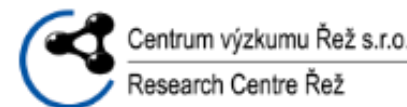
Molten Salt reactor challenges

- MSR are complex, and difficulties are multidisciplinary
- Technological challenges need to be solved for a safe and economic MSR.
- Time-consuming and costly experiments are required, to tackle these challenges and provide a basis to license MSR designs



Program overview

- Sponsored by the Dutch Ministry of Economic Affairs as part of a broader Nuclear Energy R&D program.
- In collaborations with JRC, TU Delft and CV Rez, which provide complementary competences
- Program objective: provide meaningful contribution to MSR technology development.
 - Obtain operational experience
 - Improve safety
 - Support materials development
 - Tackle waste issues
 - Integral Demonstration



Current Program and Timeline

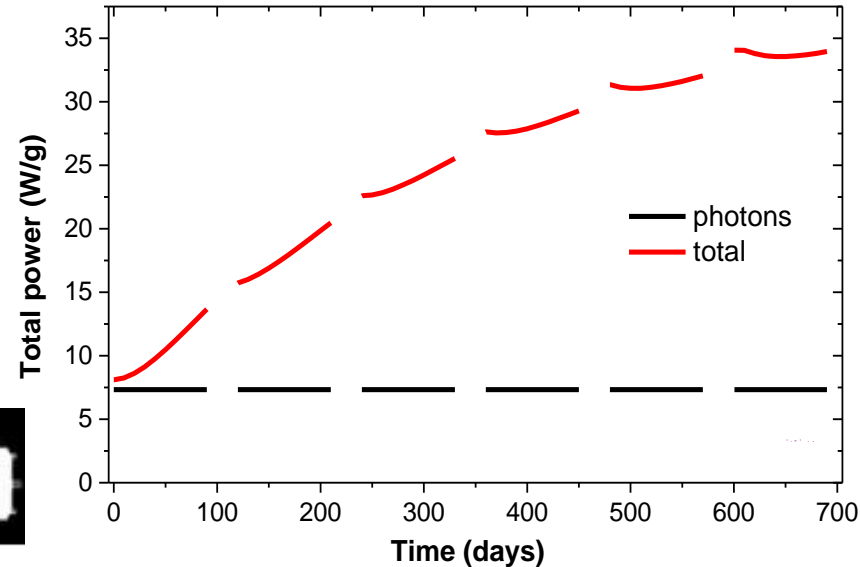
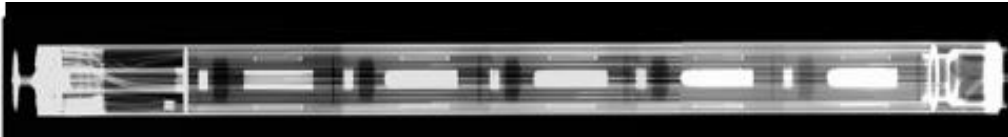
	2021				2022				2023				2024				
	Q1	Q2	Q3	Q4	Q1	Q2	Q3	Q4	Q1	Q2	Q3	Q4	Q1	Q2	Q3	Q4	
Waste	Installation oven hot cell								Waste conversion SALIENT-01								
SALIENT-01	PIE																
				Graphite infiltration													
ENICKMA-1		Irradiation				PIE								Mechanical testing			
ENICKMA-2	Feasibility				Design & realisation						(creep) Irradiation						
SALIENT-03					Irradiation / monitoring						PIE						
SALIENT-HB			Feasibility						Design & realisation						Irradiation		
SAGA (gamma)	Recombination																
					New Samples				Recombination								
	preparation								irradiation					PIE			

- Focus on irradiation technology
- Focus on generic topics (not specific for certain concepts)
- Ambitious program with limited funding, program open for partnering

SALIENT-01

Scope:

- Irradiation of ^{78}LiF - $^{22}\text{ThF}_4$ salt samples in graphite crucibles
- Determination of fission gas release
- Study noble metal deposition
- Extensive electron microscopy (SEM/WDS) to explore post-irradiation condition of the samples

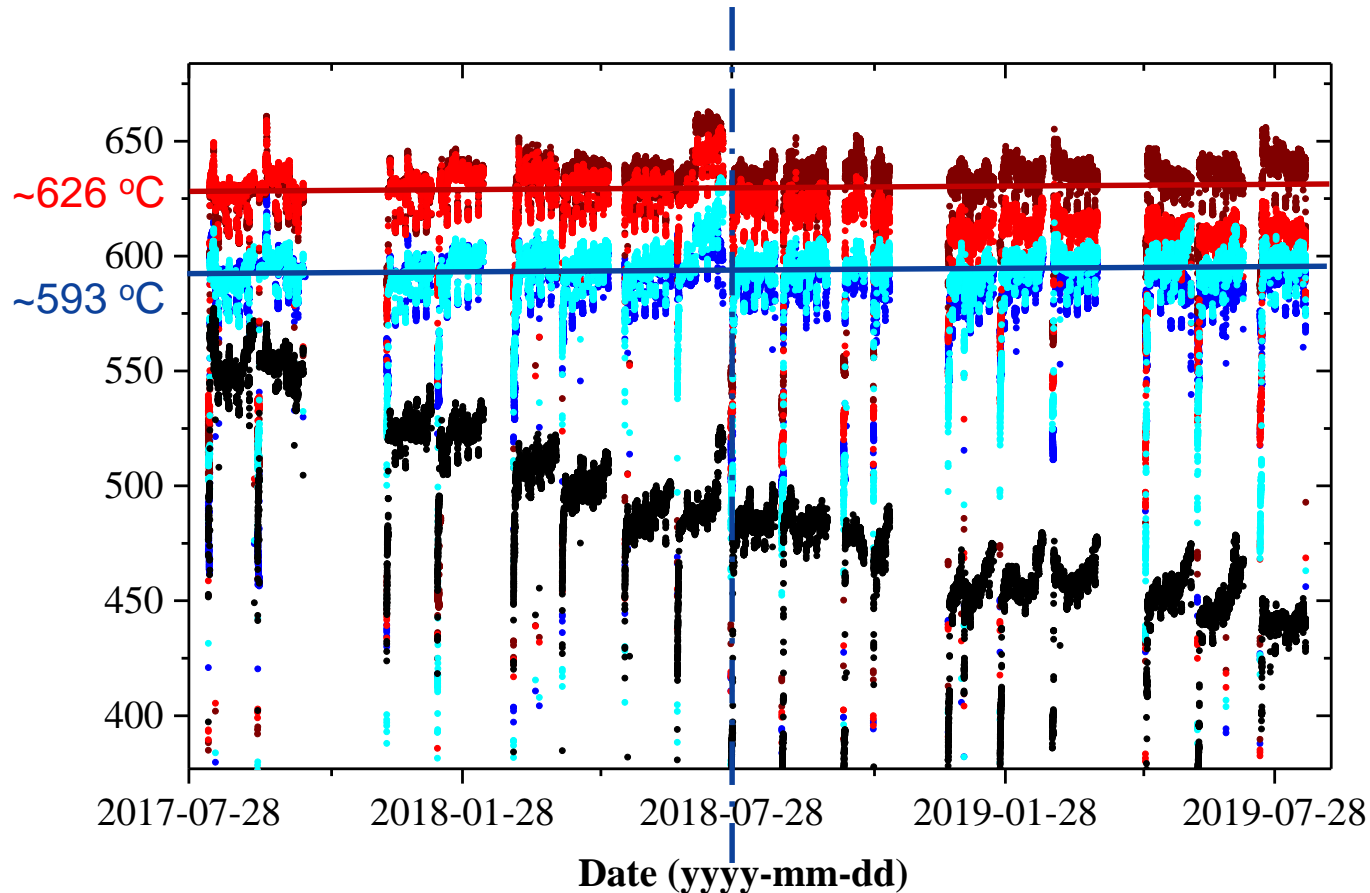


Irradiation (August 2017 – August 2019)

Measured graphite wall temperatures over 508 full power days.

Active temperature regulation of the salt-bearing capsules.

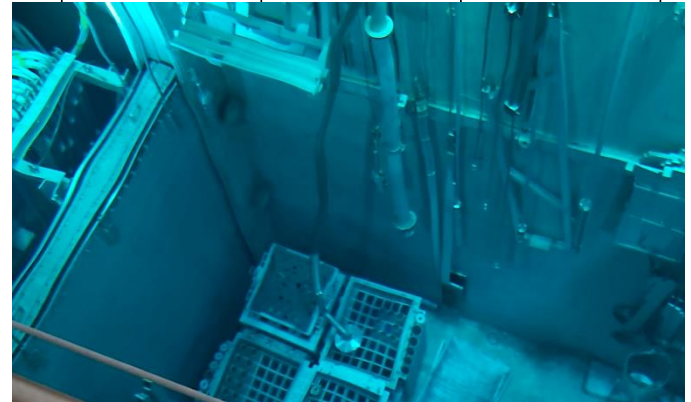
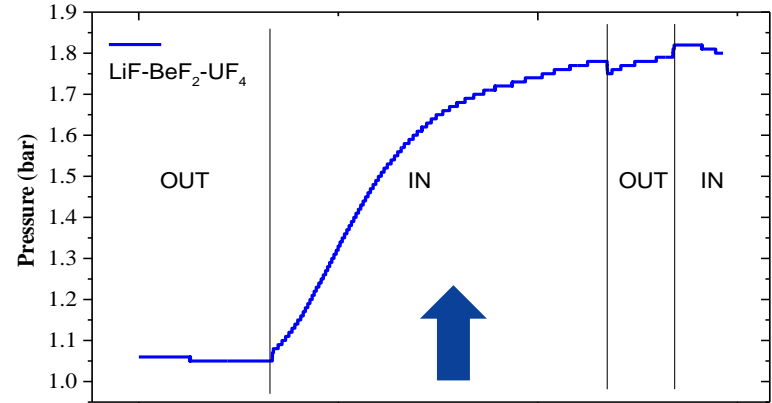
SALIENT-01 was moved to a lower-flux position following cycle 8.



Next Steps (H2020 SAMOSAFAER)

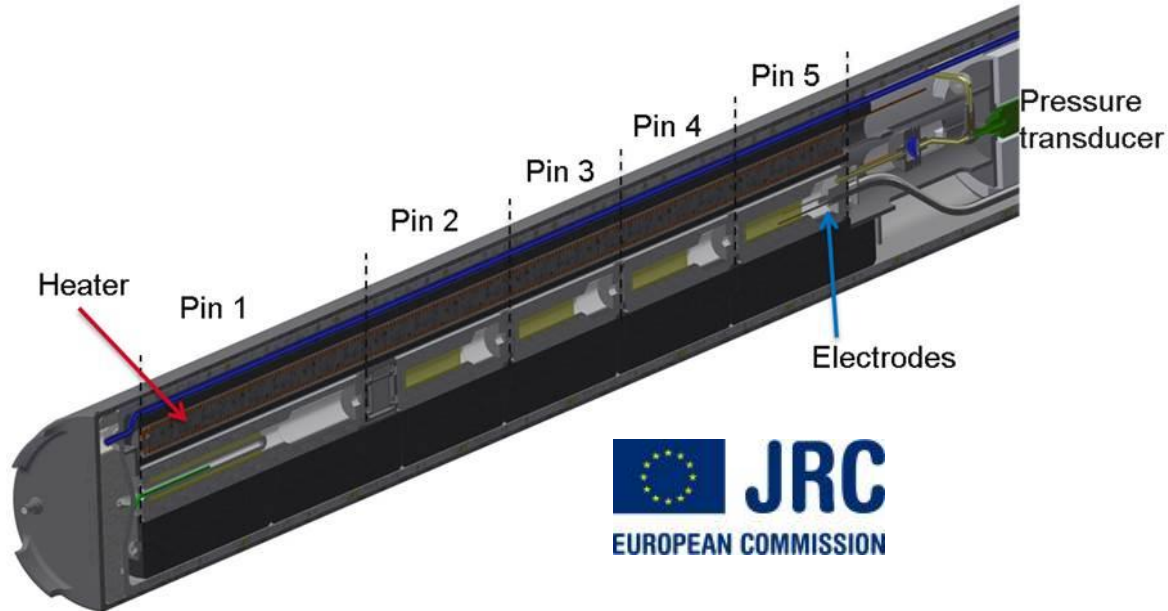
1. Plenum gas analysis - DONE
2. Transfer of SALIENT-01 to a smaller cell containing inert atmosphere
3. Removal of the 1st containment to recover the samples and activation monitoring sets
4. Preparation of small samples for transport to JRC Karlsruhe (→ Knudsen Cell Analysis)
5. Preparation of samples for in-cell microscopy (Light Microscopy and SEM/EDS/WDS)

The SAGA facility for quantifying radiolytic gas production



Irradiation tests in preparation

- ENICKMA: Helium embrittlement in nickel-based alloys (Hastelloy N, Hastelloy 242, MONICR) and ASTRID steel (316 L(N)) at 650-730 °C.
- SALIENT-03: Molten salt corrosion of Hastelloy N and fission product behavior.



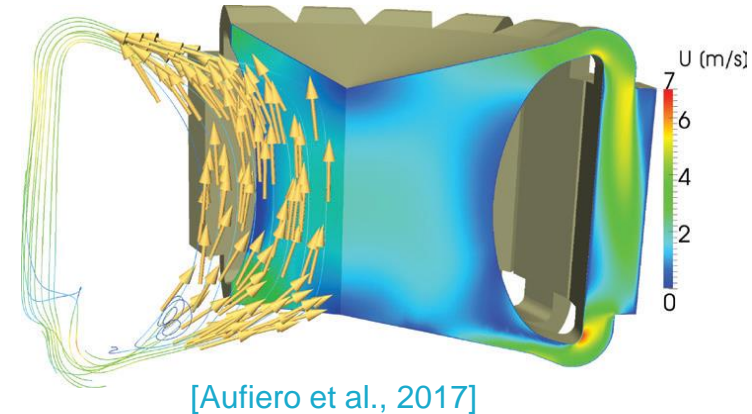
Centrum výzkumu Řež s.r.o.
Research Centre Řež



Modelling & Simulation Support

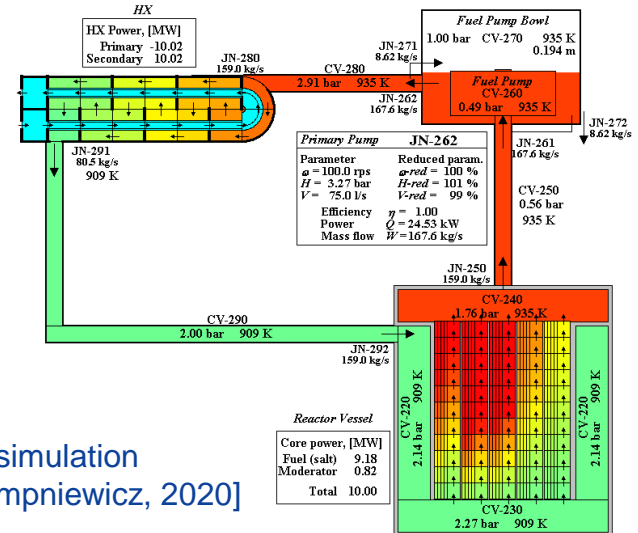
GeN-Foam

- 3D advanced multi-physics solver development based on open-source software [Fiorina et al., 2015]
 - 3D thermal-hydraulics CFD sub-solver
 - Displacement based thermal-mechanics sub-solver
 - Multi-group neutron diffusion sub-solver
 - Finite-difference sub-scale fuel model



1D fast running SPECTRA system code

- Specific MSR features:
 - delayed neutron precursor drift
 - fission product transport in (fueled) molten salt reactors
 - noble gas and noble metal behavior
 - noble metal extraction
 - chromium leaching and deposition

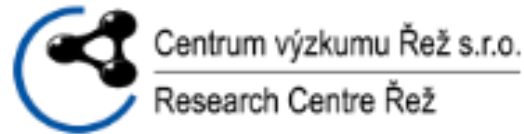


MSRE simulation
[Roelofs & Stempniewicz, 2020]

Acknowledgements



Ensuring Nuclear Performance



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